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Comments and questions should be directed to the NRC Clearance Officer, Brenda Jo Shelton, U.S. Nuclear Regulatory Commission, T–6 F33, Washington, D.C., 20555–0001, (301) 415–7233, or by Internet electronic mail at BJS@NRC.GOV.

Dated at Rockville, Maryland, this 28th day of November, 1995.

For the Nuclear Regulatory Commission.

Gerald F. Cranford,

Designated Senior Official for Information Resources Management.

[FR Doc. 95–29424 Filed 12–1–95; 8:45 am]

BILLING CODE 7590–01–P

[Docket Nos. 50–344 and 50–412]

Duquesne Light Company, Ohio Edison Company, Pennsylvania Power Company, The Cleveland Electric Illuminating Company, The Toledo Edison Company, Beaver Valley Power Station, Units 1 and 2; Notice of Partial Withdrawal of Application for Amendment to Facility Operating License

The United States Nuclear Regulatory Commission (the Commission) has granted the request by Duquesne Light Company (the licensee) to withdraw a portion of its September 13, 1995, application for a proposed amendment to Facility Operating License Nos. DPR–66 and NPF–73 for Beaver Valley Power Station, Units 1 and 2 (BVPS–1 and BVPS–2), located in Beaver County, Pennsylvania.

The proposed amendment involved revision of the Administrative Controls section (Technical Specifications (TS) 6.8.6.a.2), 6.8.6.a.7), and 6.14.a.2)) and the Bases section for TS 3/4.11.1.4 of the BVPS–1 and BVPS–2 TSs to be consistent with the requirements of the Offsite Dose Calculation Manual (ODCM). The ODCM was recently updated to reflect the radioactive liquid and gaseous effluent release limits and the liquid holdup tank activity limit of BVPS–1 License Amendment No. 188 and BVPS–2 License Amendment No. 70 which were issued June 12, 1995.

On October 16, 1995, the licensee submitted a letter to the NRC requesting withdrawal of the proposed changes to TS 6.14.a.2) and to the Bases for TS 3/4.11.1.4 because these changes incorrectly referenced superseded sections of 10 CFR Part 20.

The Commission has previously issued a Notice of Consideration of Issuance of Amendments to Facility Operating Licenses, Proposed No Significant Hazards Consideration Determination, and Opportunity for a Hearing, which was published in the Federal Register on September 22, 1995 (60 FR 49292).

For further details with respect to this action, see the application for amendment dated September 13, 1995, and the licensee's letter of October 16, 1995, which withdrew the portion of the application for license amendment. The above documents are available for public inspection at the Commission's Public Document Room, 2120 L Street, NW., Washington, DC 20555 and at the B.F. Jones Memorial Library, 663 Franklin Avenue, Aliquippa, Pennsylvania 15001.

Dated at Rockville, Maryland, 21st day of 1995.

For the Nuclear Regulatory Commission.

Donald S. Brinkman,

Senior Project Manager, Project Directorate I–2, Division of Reactor Projects—I/II, Office of Nuclear Reactor Regulation.

[FR Doc. 95–29423 Filed 12–1–95; 8:45 am]

BILLING CODE 7590–01–P

[Docket No. 50–245]

Northeast Nuclear Energy Company; Notice of Consideration of Issuance of Amendment to Facility Operating License, Proposed No Significant Hazards Consideration Determination, and Opportunity for a Hearing

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License No. DPR–21 issued to Northeast Nuclear Energy Company (NNECO) for operation of the Millstone Nuclear Power Station, Unit 1, located in Waterford, Connecticut.

The proposed amendment would remove the Limiting Condition for Operation (LCO) and Surveillance Requirements for the loss-of-normal power (LNP) trip function from Tables 3.2.2 and 4.2.1 and insert new LCO 3.2.F and Surveillance Requirement 4.2.F. In addition, the proposed amendment will add a new table to specify the required LNP instrumentation for each bus, will update the Table of Contents, will make some editorial changes, and will revise the associated Bases section.

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

The Commission has made a proposed determination that the amendment request involves no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

NNECO has reviewed the proposed change against the criteria set forth in 10 CFR 50.92 and has concluded that the change does not involve a Significant Hazards Consideration (SHC). The bases for this conclusion are that the three criteria of 10 CFR 50.92(c), discussed separately below, are not compromised. The proposed change does not involve an SHC because the change would not:

1. Involve a significant increase in the probability or consequences of an accident previously analyzed:

These changes do not increase the probability of a loss of offsite power event or the occurrence of any accidents which assume loss of offsite power. This is ensured by the LNP instrumentation system design which uses multiple sensing relays and qualified Class 1E components, as well as conservative operability and surveillance requirements.

The LNP instrumentation for a safety division consists of discrete voltage sensing, time delay, initiation, and auxiliary logic relaying. The LNP instrumentation for a safety division is a single trip system (initiation channel) controlled by two instrument channels. Each instrument channel consists of a loss of voltage trip function and a degraded voltage trip function. The two instrument channels each provide a trip signal. The LNP trip signal is comprised of two instrument channels made up of a loss of voltage relay and its timer, and a degraded voltage relay and its timer. The signals from the two instrument channels feed into a delay timer, producing the LNP initiation system for the safety division. The S1 LNP instrumentation is powered from the